

November 9, 2004

Mr. David A. Christian
Sr. Vice President and Chief Nuclear Officer
Dominion Nuclear Connecticut, Inc.
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: REVIEW OF STEAM GENERATOR TUBE INSERVICE INSPECTION REPORT
FOR THE 2003 REFUELING OUTAGE AT MILLSTONE POWER STATION,
UNIT NO. 2 (TAC NO. MC2525)

Dear Mr. Christian:

By letters dated November 5, 2003, as supplemented by letters dated February 26, 2004 and September 23, 2004, Dominion Nuclear Connecticut, Inc. (DNC), submitted reports summarizing the steam generator tube inservice inspections performed at Millstone Power Station, Unit No. 2 (MP2) during Refueling Outage 15 in October 2003.

As discussed in the enclosed evaluation, the Nuclear Regulatory Commission (NRC) staff concluded that DNC provided the information required by the Technical Specifications for MP2, and the NRC staff did not identify any technical issues that warranted follow-up action at this time. If you have any questions, please contact me at (301) 415-1484.

Sincerely,

/RA/

Victor Nerses, Senior Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-336

Enclosure: As stated

cc w/encl: See next page

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UNITED STATES NUCLEAR REGULATORY COMMISSION
EVALUATION OF 2003 REFUELING OUTAGE
STEAM GENERATOR TUBE INSERVICE INSPECTION RESULTS
MILLSTONE POWER STATION, UNIT NO. 2
DOCKET NO. 50-336

By letters dated November 5, 2003, as supplemented by letters dated February 26, 2004 and September 23, 2004, Dominion Nuclear Connecticut, Inc. (licensee) submitted information to the Nuclear Regulatory Commission (NRC) pertaining to their 2003 steam generator (SG) tube inspection at Millstone Power Station, Unit No. 2 (MP2).

The two steam generators at MP2 were replaced in 1993 with SGs fabricated by Babcock and Wilcox International. Each SG nominally contains 8,523 thermally treated Alloy 690 tubes. Each tube has a nominal outside diameter of 0.750-inch and a nominal wall thickness of 0.0445-inch. The tubes were hydraulically expanded at both ends for the full length of the tubesheet and are supported by a number of stainless steel tube support plates. The U-bends of the tubes installed in rows 1 through 8 were thermally stress relieved after bending.

The licensee provided the scope, extent, methods, and results of their SG tube inspections in the documents mentioned above. In addition, the licensee described corrective actions (e.g., tube plugging or repair) taken in response to the inspection findings. Based on a review of the information provided, the NRC staff concludes that the licensee provided the information required by their technical specifications. In addition, the staff concludes that there are no technical issues that warrant follow-up action at this time, since the inspections appear to be consistent with the objective of detecting potential tube degradation, and the inspection results appear to be consistent with industry operating experience at similarly designed and operated units. The staff has the following observations regarding the information provided.

The licensee has detected a few bulges, dents, and dings in their steam generators. Although the submitted reports were not clear as to whether some of these indications were service induced (i.e., were not present in the baseline inspection based on either reports from the baseline inspection or a subsequent review of the baseline data), the licensee provided inspection results from these locations which have shown no degradation affecting these locations.

Enclosure

Millstone Power Station, Unit No. 2

cc:

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